

# **From Beam-target to Thermonuclear Fusion in the Dense Plasma Focus Pinch: Energy throughput scaling**

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# Outline of talk:

## I: Background concepts:

- Electromagnetic drive, MRN and typical speeds, Speed factor  $S$ .
- Mach  $\gg 1$  driven plasmas, temperature vs speed
- Cross-sections for nuclear fusion: beam-target and thermonuclear

# Outline of talk:

## II: DPF Fusion: Beam-target predominance: why? - Throughput scaling

- Inductive voltages generate tens of keV, driving beam ions to 100 keV energy – optimum fusion x-section
- Shock speed generates  $< 0.5$  keV kinetic temperature; fusion x-section  $10^{-13}$  times below optimum – negligibly small
- Hence Beam-target fusion mode predominates
- Throughput (Output/Input) Scaling to break-even  $Q = 1$
- Breakeven point found through numerical experiments

**Note: Discuss** Gross PF pinch (scalable), **NOT** hot spots

## Outline of talk:

### III: Transitioning to thermonuclear mode

- How?
- Optimum conditions for neutron yield in thermonuclear mode
- Throughput (Output/Input) Scaling to break-even  $Q = 1$
- Breakeven point found through numerical experiments

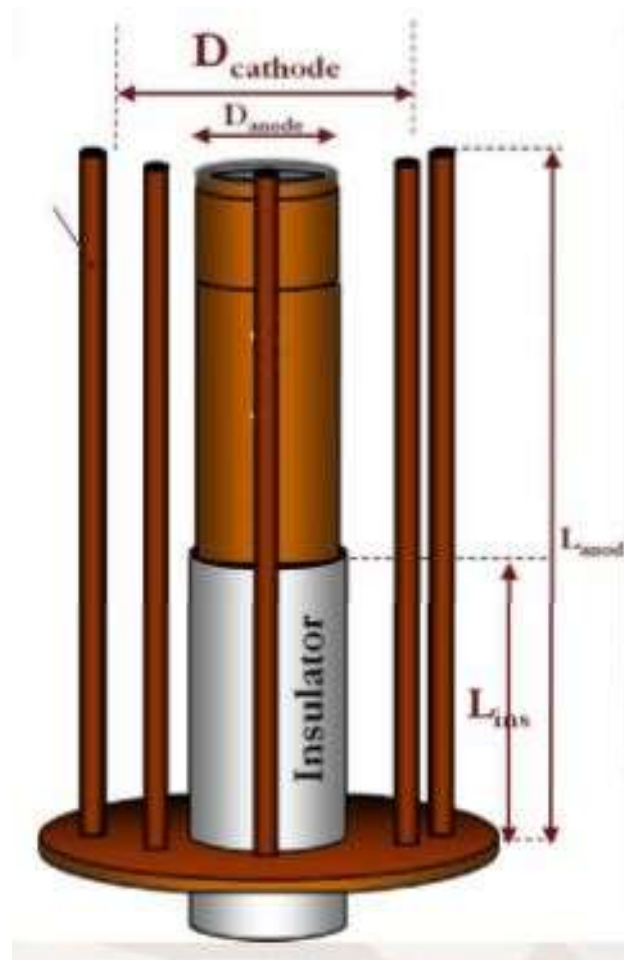
# Outline of talk:

## IV: Comparison:

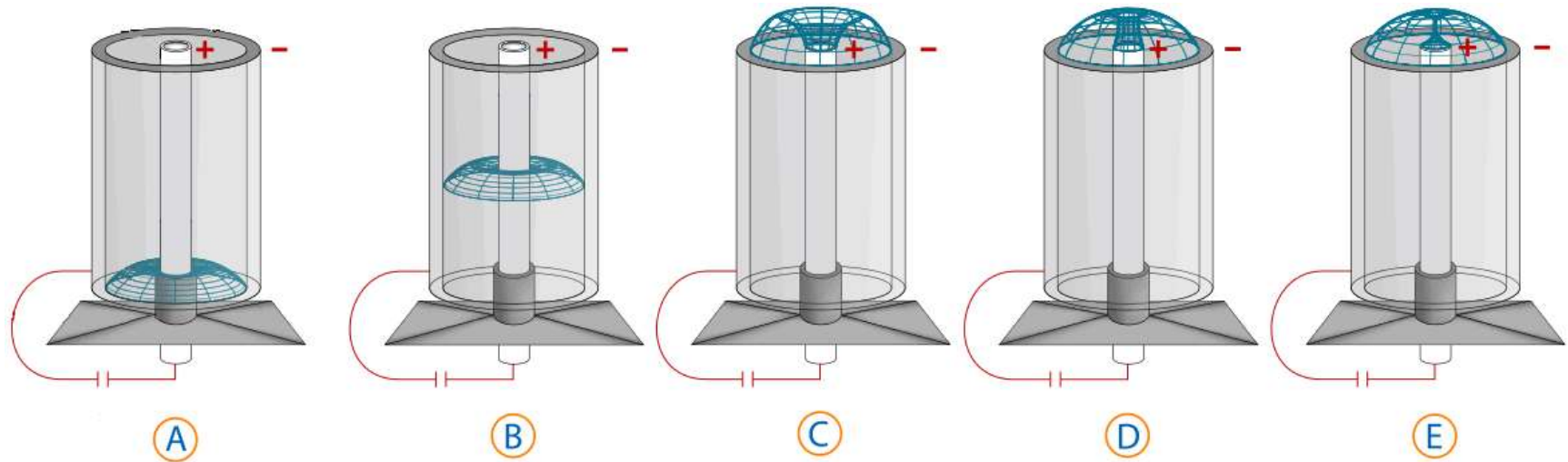
### **Beam-target (DPFQ1) breakeven point versus Thermonuclear breakeven point**

- **Proposing a feasible test point DPF0.01**
- **Conclusions**

# Some introductory slides: DPF electrodes — (from Ali Abdou)

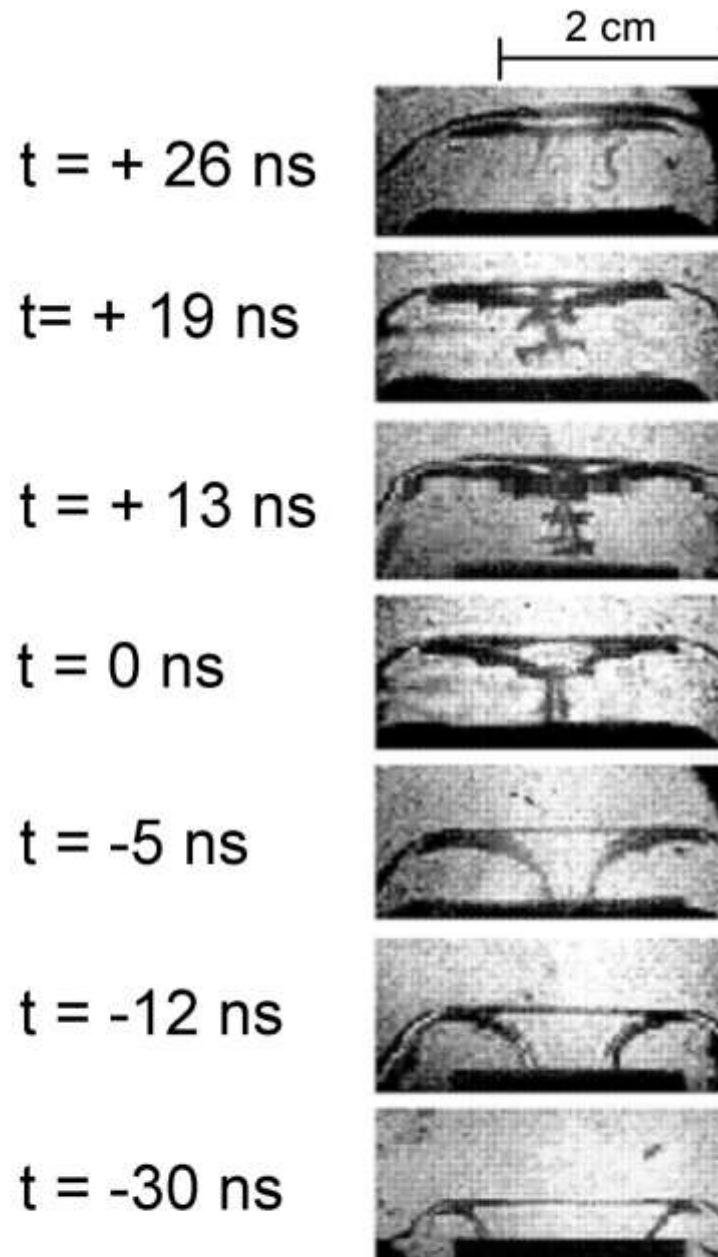


# DPF dynamics – axial phase, radial phase – from internet



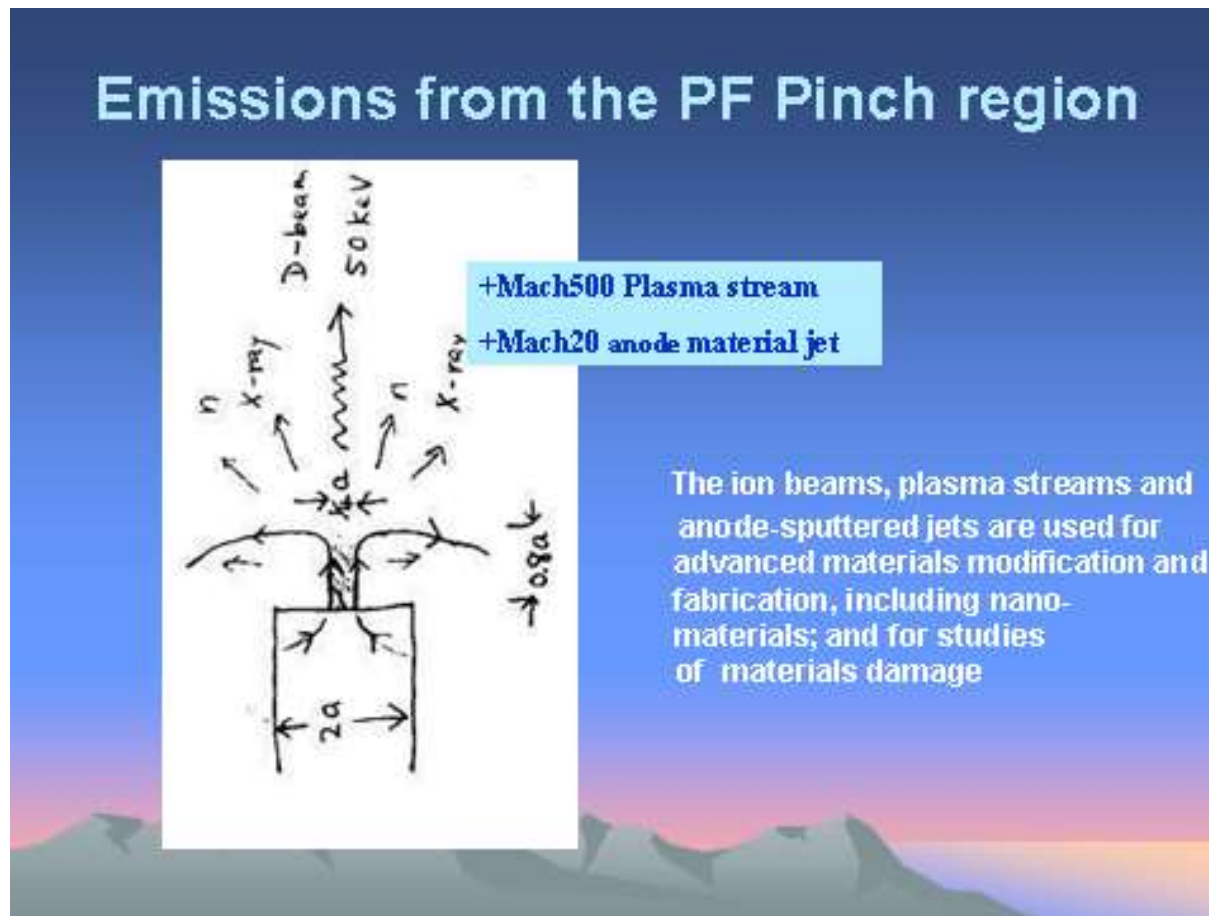
# Radial dynamics – confirmed by our ns Shadowgraphs –

(from M Shahid Rafique, PhD thesis, NTU, Singapore [1])

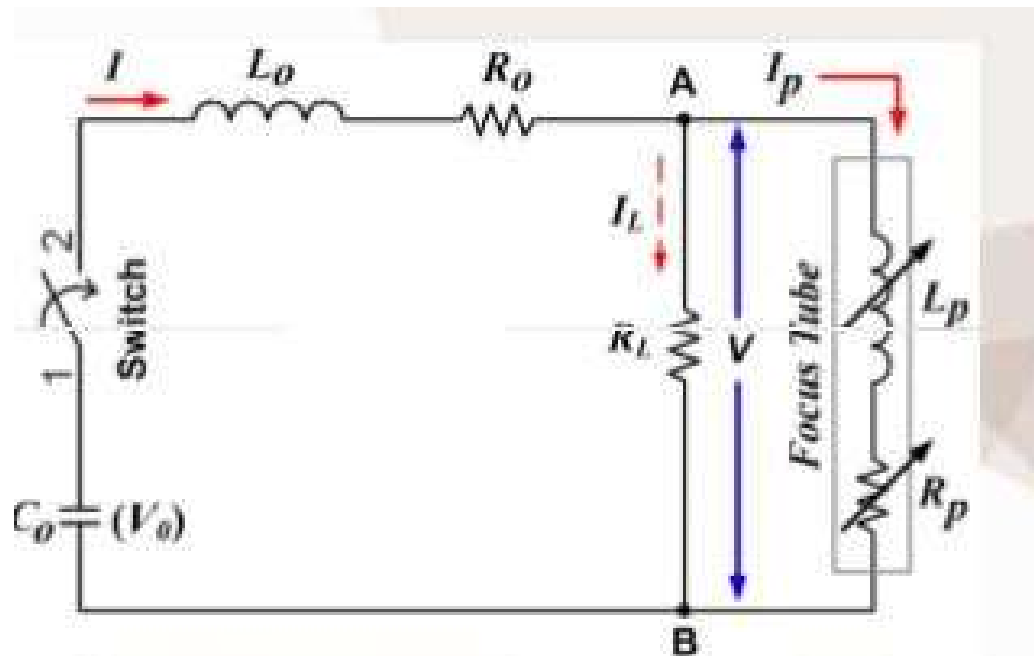




Among radiations from DPF pinch are the fusion neutrons- 50%-50% D-T has the best cross-section for nuclear fusion; producing 14.1 MeV neutrons

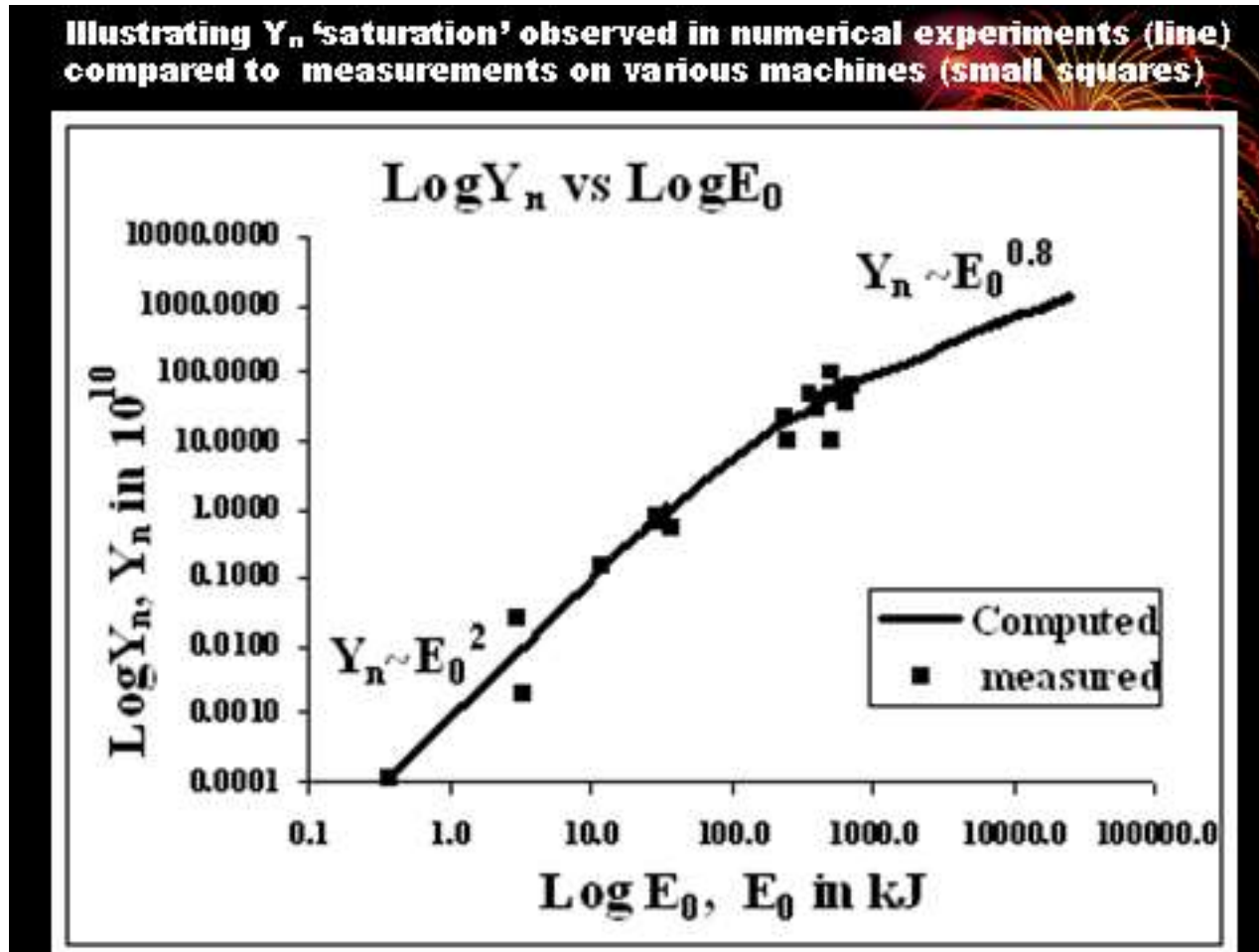


# DPF circuit representation



**DPF Electric Circuits**

Initial hope:  $Y_n \sim E_0^2$  [2], suggesting breakeven at hundreds MJ .  
 However dominance of dynamic resistance for very large capacitor banks causes current- scaling deterioration [3], leading to yield- scaling deterioration to  $Y_n \sim E_0^{0.8}$  – suggests **no break-even unless** capacitor voltages are greatly increased [4] aided by increase in pressure [4].



# **I: Some background concepts for DPF:**

## **1. Electromagnetic drive:**

- Interaction of electric current and magnetic field (JXB) produces high plasma speeds and temperature. Electromagnetic drive is efficient when the magnetic Reynolds number  $M_{RN}$  is high.

## 2. Magnetic Reynold's number $M_{RN}$ and speed factor $S$ :

- For electromagnetic em drive to be efficient, the condition is that  $M_{RN} \gg 1$ .
- For high Mach shock waves we have shown [5]  $M_{RN} \sim v^4$ ; with transition point around 5 cm/ $\mu$ s for D-T plasma.
- $M_{RN} \gg 1$  is fulfilled in the DPF because plasma speeds are highly supersonic.
- For electromagnetic drive, the speed is governed by a fundamental factor  $S = (I/a) / \rho^{0.5}$

Over whole **range of existing DPF's (sub kJ to MJ)**

$S \sim 70 - 200$ , practically constant [6]

- $S = (I/a) / \rho^{0.5}$

Value of S: typically 100 (kA/cm) Torr<sup>0.5</sup>

(I/a) : Typically **200 kA/cm**

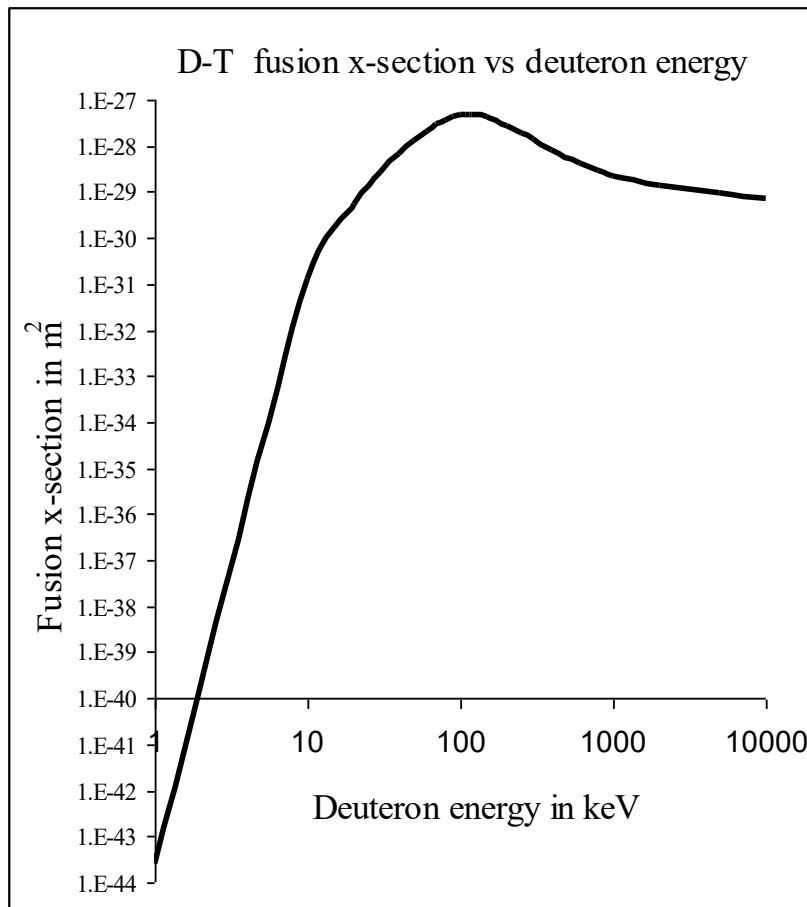
Operational pressure: Typically 4 Torr (~ **0.01 atm**)

### 3. High Mach (Mach $\gg 1$ ) shock waves:

- A convenient unit of speed for electromagnetically driven systems is cm/ $\mu$ s. (1 cm/ $\mu$ s =  $10^4$  m/ $\mu$ s  $\sim$  Mach 10 in D-T).
- In DPF's, speeds are typically 10 cm/ $\mu$ s and higher. Thus DPF plasma drives are characterized by Mach  $> 100$  supersonic shock waves.
- Shock wave systems are equi-partitioned with approximately equal amounts of energy in the thermal modes and the kinetic components.
- The shock conservation equations enable the plasma temperature  $T$  to be computed [6a] from the shock speed  $v$ .
- For a 50%-50% D-T shock system:

$$T = 2.8 \times 10^{-5} v^2$$

## 4. Cross-sections for D-T nuclear fusion : applicable to Beam- (plasma) target:



- 10 keV  $\sigma$  is  **$10^{13}$**  higher than  $\sigma$  at 1 keV.
- 10 keV to peak  $\sigma$  at  $\sim 100$  keV, another increase of  **$10^3$** .
- Optimum  $\sigma$  occurs at  $\sim 100$  keV.
- Higher beam energy  $\sigma$  drops.  
1 MeV  $\sigma$  **has dropped almost 100x**

[7] S Glasstone and R H Lovberg, Controlled Thermonuclear Reactions. D Van Nostrand Company, New Jersey (1960)

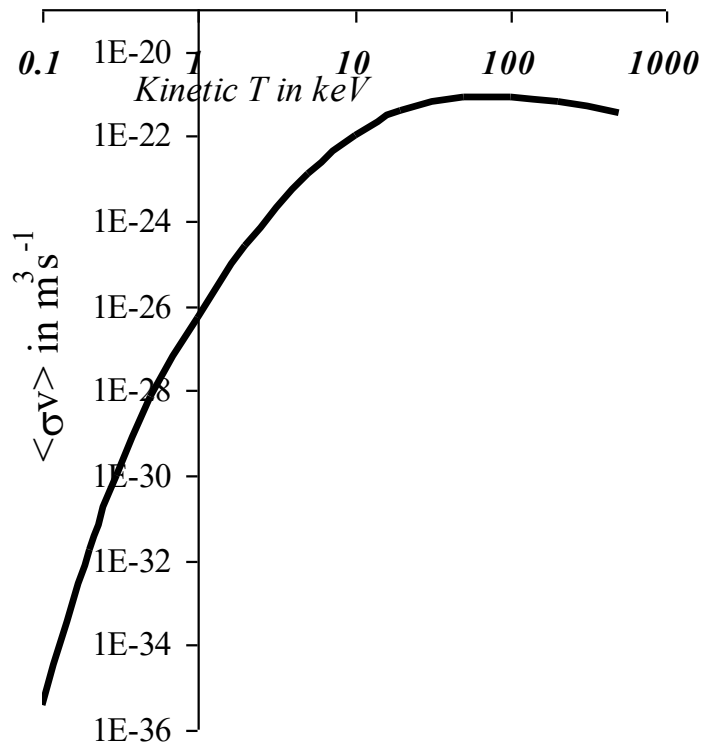
[8] J.D.Huba. 2006 Plasma Formulary pg44 [Http://wwwppd.nrl.navy.mil/nrlformulary/NRL\\_FORMULARY\\_07.pdf](http://wwwppd.nrl.navy.mil/nrlformulary/NRL_FORMULARY_07.pdf)



## 5. Cross-sections for nuclear fusion: Thermonuclear:

Relevant cross-section is the  $\langle\sigma v\rangle$  ie product of cross-section and particle speed  $v$  averaged over Maxwellian distribution

$\langle\sigma v\rangle$  vs kinetic temperature  $kT$



- From 0.1 to 1 keV  $\langle\sigma v\rangle$  **increases  $10^9$** .
- From 1 to 10 keV  $\langle\sigma v\rangle$  **increases  $10^4$** .
- From 10 keV, the  $\langle\sigma v\rangle$  increases  $<10$  to its peak value at 70 keV.
- From  $\langle\sigma v\rangle$  point of view, good operational point is around **70 keV**

[7] [8]

## II: Present-generation DPF (sub kJ to MJ) operate predominantly in beam-target mode: why?

- All present DPF's (sub kJ to MJ) operate with same speed: axial around 10 cm/ $\mu$ s and radial around 20 cm/ $\mu$ s [6].
- This gives a temperature in D-T of  $\sim 0.3 \times 10^6$  K for the axial phase plasma and  $\sim 1.2 \times 10^6$  for the radial phase on axis shock; to about 2.4 million K in the stagnated plasma column on shock reflection on-axis.
- The gross pinch **temperature is typically < 0.5 keV** (1 keV =  $1.14 \times 10^7$  K) — very low temperature from fusion point of view.
- Resulting from highly supersonic piston action, inductive voltages (back EMF motor effect, if we recall) typically 20 – 40 kV are generated, producing **60 – 120 keV ions**. These energies are near optimum from fusion point of view.
- **Thermonuclear yield: low; Beam – target yield: high**
- **Advantage: low investment in plasma energy vs optimum beam energy**

# **B-t fusion scaling: output fusion energy to input energy.**

- We ask the question: How much D-T fusion energy (from beam-target) do we get per unit energy needed to drive the DPF?
- We first ask the question: How many D-T neutrons (from beam-target) do we get per unit pinch energy.

Modelling by the Lee code provides the number of beam-target neutrons [9-11] as follows:

- $$Y_{b-t} = C_n n_i I_{pinch}^2 z_p^2 (\ln(b/r_p)) [\sigma/V_{max}^{1/2}]$$

pinch radius  $r_p$ , pinch length  $z_p$ , pinch ion density  $n_i$  driven by pinch current  $I_{pinch}$ ; with beam ion energy  $V_{max}$  and corresponding beam-target fusion cross-section  $\sigma$ .

Here all quantities are in SI units with  $V_{max}$  in keV and the constant  $C_n = 1.4 \times 10^7$  (calibrated)

The pinch energy [12] at temperature T is

- $E_{pinch} = [kT / (\gamma - 1)] n_i (1 + Z_{eff}) \pi r_p^2 z_p$

k = Boltzmann constant,  $\gamma$  = specific heat ratio = 5/3 and  $Z_{eff} = 1$  (for fully ionized D-T plasma).

- Assume an equilibrium pinch, equate the confining magnetic pressure to the hydrostatic plasma pressure. Thus:

- $I_{pinch}^2 = 2\pi \times 10^7 n_i kT (1 + Z_{eff}) r_p^2$

Divide  $Y_{b-t}$  by  $E_{pinch}$ , replacing  $I_{pinch}^2$  in  $Y_{b-t}$  by the RHS of Eq (4) we get the required number of beam-target neutrons per unit pinch energy.

Note:  $(\ln(b/r_p)) \sim 2$  and  $z_p \sim 1.4$  a ([6] for fully ionized hollow anode DPF)

## General scaling for number of D-T DPF beam-target neutrons

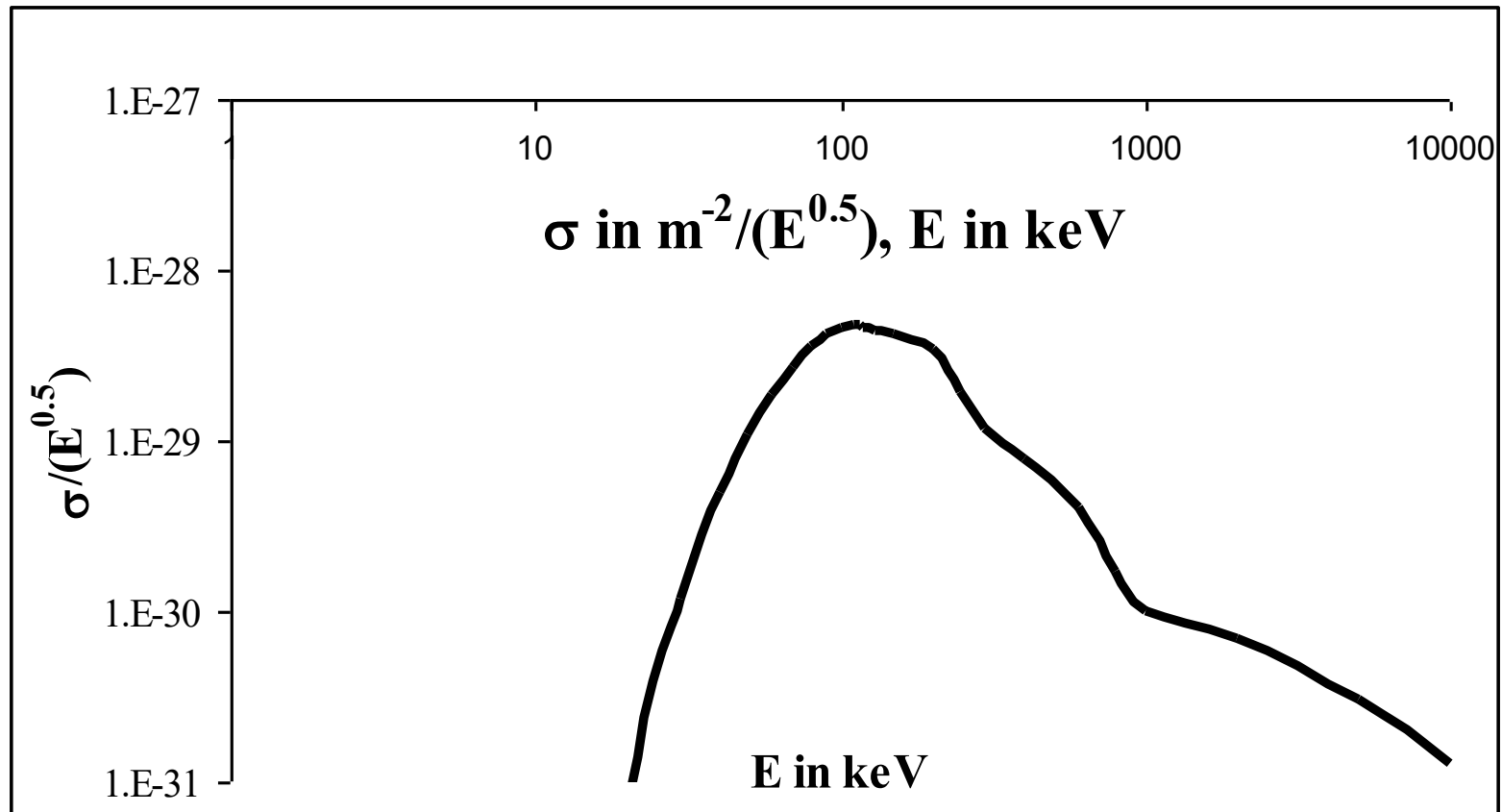
- $Y_{b-t} / E_{pinch} = 5 \times 10^{14} n_i a [\sigma / V_{max}^{1/2}]$

This general scaling stipulates that the number of beam-target neutrons depends on the pinch ion density, the anode radius ‘a’ and the energy of the D-T beam ions through the fraction  $[\sigma / V_{max}^{1/2}]$ .

# Optimising beam – target scaling by choosing optimum beam energy through factor $[\sigma/V_{max}^{1/2}]$

**Optimum value** of  $[\sigma/V_{max}^{1/2}]$  occurs at  $V_{max} = 115$  keV with value  **$5 \times 10^{-29} \text{ m}^2$** .

Note: **broad flat maximum**, in range of beam ion energy 90 – 120 keV



Operate the DPF at **optimum deuteron beam energy around 100 keV** then the scaling for  $Y_{b-t} / E_{pinch}$  is optimized:

- $(Y_{b-t} / E_{pinch})_{100keV} = 2.5 \times 10^{-14} n_i a$

A D-T neutron has energy of 14.1 MeV ie  $2.26 \times 10^{-12}$  J ; also estimating that  $E_{pinch} \sim 10\%$  of the stored energy  $E_0$ , then we have the ratio

$$Q_{100keV} = (E_{b-t} / E_0)_{100keV} = \underline{6 \times 10^{-27} n_i a}$$



# Better than break- even

for  $Q > 1$

$$\underline{6 \times 10^{-27} n_i a > 1}$$

Example: ' $a$ ' = **1 m** then for  $Q > 1$

$$n_i > 1.7 \times 10^{26} \text{ m}^{-3} \quad (100 \text{ keV})$$

(ie **6 atm** of fill pressure is sufficient for break-even).

# Numerical experiments - disagree

- In fact thousands of runs have been made using the Lee code [9,11] in DPF's of meter-size anode radius at beam-target conditions (following present generation DPF's with axial speed around  $10 \text{ cm}/\mu\text{s}$ ) – over years!
- Nothing approaching anywhere near break-even has been observed at such DPF sizes and densities; and even bigger sizes and densities!

**Discrepancy**- is due to **condition of 100 keV beam ion** energy, which is **not met**

$$V = \frac{\mu}{2\pi} \left[ (\ln c) z_0 + \left( \ln \frac{b}{r_p} \right) z_f \right] f_c \frac{dI}{dt} + \frac{\mu}{2\pi} \left[ \left( \ln \frac{b}{r_p} \right) \frac{dz_f}{dt} - \frac{z_f}{r_p} \frac{dr_p}{dt} \right] f_c I$$

**V** = **V1** + **V2**  
**V** = **V1** + **V21** + **V22**

	Tube	dI/dt*position	speed*I			Effective
	Voltage kV	term	term	dz/dt term	drp/dt term	Beam ion
	<b>V</b>	<b>V1</b>	<b>V2</b>	V21	V22	Energy keV
NX2 <b>15kV</b>	<b>25.1</b>	<b>-10.6</b>	<b>35.7</b>	17.1	18.6	<b>75</b>
PF1000 <b>27kV</b>	<b>25.7</b>	<b>-29.8</b>	<b>55.5</b>	29.1	26.4	<b>77</b>
PF1000 <b>27kV</b> RESF=0.1	<b>57.4</b>	<b>-65.6</b>	<b>123</b>	62.9	59.7	<b>172</b>
25MJ <b>35kV</b>	<b>62</b>	<b>-285</b>	<b>347</b>	185	162	<b>186</b>
162MJ <b>90kV</b>	<b>106</b>	<b>-484</b>	<b>590</b>	308	282	<b>318</b>
16200MJ <b>900kV</b>	<b>1460</b>	<b>-6121</b>	<b>7580</b>	3970	3610	<b>4380</b>

Scaled up DPF requires 500 MA. The dynamic impedance of  $2\text{m}\Omega$  requires **bank voltage of 1 MV**;

Thus induced beam ion energy  $\sim 5\text{ MeV}$

- For b-t DPF's the values of speeds are kept at low levels of **10 cm/ $\mu\text{s}$**  for axial and 20 cm/ $\mu\text{s}$  for the radial phases. The speed factor  $S = (I/a) / \rho_0^{1/2}$  needs to be kept at around **100**. Thus as 'a' increases, I increases proportionally; so with  $(I/a) = 200\text{ kA per cm anode radius}$ , we need **20 MA for 1 m anode radius**. The increase in **operational pressure to atm** will require a further increase of current to around 500 MA.
- **Thus the scaled-up m-sized DPF will have induced voltages increased to multiple MV; way past optimum**  $[\sigma/V_{max}^{1/2}]$ . Indeed at **10 MeV** this cross-section parameter has **dropped by 1000 times** from optimum value.
- These estimates provide a guide for numerical experiments for a scaled-up DPF, to obtain an operational point for a B-T device; although we know the beam ions will be far too energetic.

# Beam-target numerical experiment (1.2 MV, 5 atm) generating $Q = 0.002$

Lo	Co	b	a	zo	ro mOhm
	20	10000	120	100	60
massf	currf	massfr	currfr	Model Parameters	
	0.08	0.7	0.2	0.7	
Vo	Po	MW	A	2	Operational
	1200	4000	5	1	2Parameters

RES	F	c=b/a	$I_{peak}$	$T_p$	$v_a$	$v_s$	$v_p$	$r_{min}$	$z_{max}$	pinch $\tau$	$V_{max}$	$n_i$ pinch
kJ			kA	$10^6$ K	cm/ $\mu$ s	cm/ $\mu$ s	cm/ $\mu$ s	cm	cm	ns	kV	$10^{23}/m^3$
7.20E+06	0.11	1.20	3.5E+05	0.11	8.3	8.5	6.8	30.0	161.7	6527	1,337	1051

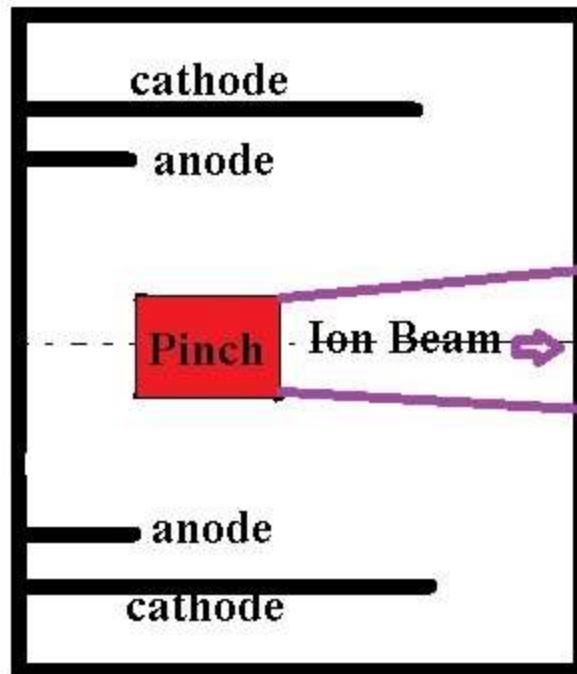
$Y_n$	EINP	SF	ID	$E_N$ kJ	$E_N/E_0$	$Y_{th}$	$Y_{b-t}$	$Y_n$
$10^{10}$	%		kA/cm					
6.1E+08	43.4	55	3476	1.4E+04	0.002	5.0E-12	6.1E+18	6.1E+18

Beam ion energy 4 MeV; number of ions in beam =  $1.3 \times 10^{22}$   
 $[\sigma/V_{max}^{0.5}]$  is down from optimum value by almost 1000 times

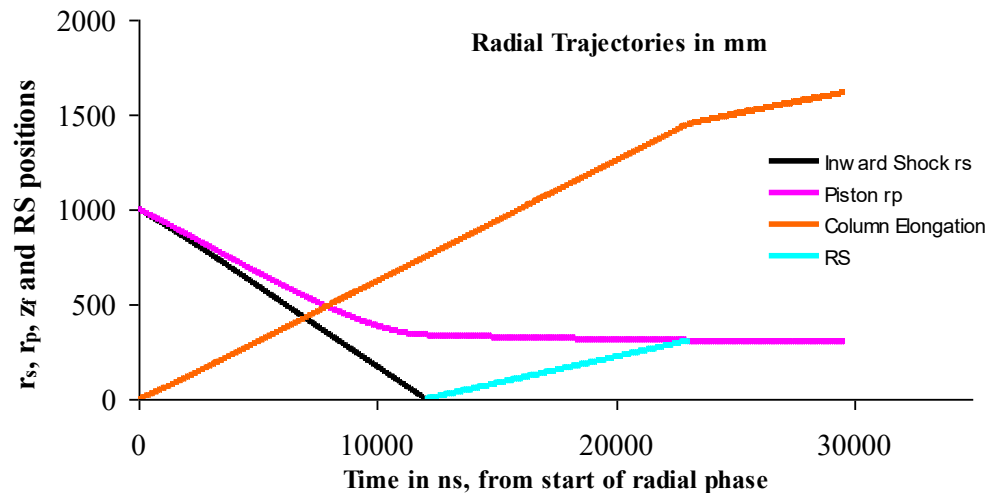
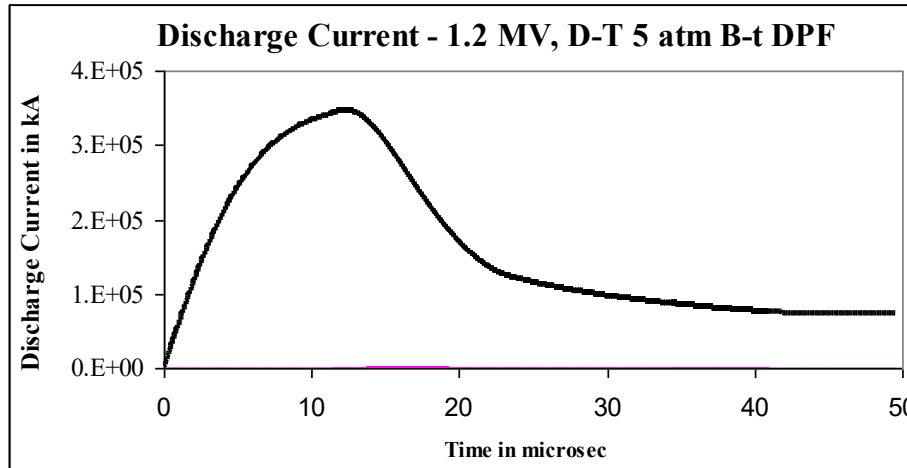
SIRIM estimates by M Akel [13] indicate **1 m path in 5 atm D-T sufficient to slow Deuteron- Triton Beam to 100 keV.**

In schematic shown below the ion beam will be moderated to 100 keV in its path ( $> 1\text{m}$ ) before leaving the chamber.

**In this schematic  $Q > 1$  enhancing the Lee Code calculation of  $Q = 0.002$ .**



# Discharge current waveform and radial trajectories of 1.2 MV, 5 atm, $a = 1\text{m}$ ; D-T DPF



# III: Transitioning to thermonuclear mode

- To operate the DPF pinch at thermonuclear conditions for D-T the pinch temperature needs to be increased from the sub-keV of present-day DPF's to  $\sim 70$  keV
- at which the optimum thermalized cross-section  $\langle \sigma v \rangle$  for D-T fusion occurs.
- This requires faster plasma speeds than presently used.
- How fast?



# III: Transitioning to **thermonuclear mode** - how?

- How to get to such high temperature?
- The primary mechanism in the DPF is the electromagnetic drive generating high plasma speeds. Thermonuclear regime is 50 times hotter; requires shock speed 7 times higher.
- Radial shock speed needs to increase to **150 cm/ $\mu$ s.**
- Speed factor S needs to increase to **1000** (from present day 100)
- Density is  $\sim 1000$ x higher. (as will be seen)
- Hence current per unit anode radius need to increase to 60 MA/cm (from present-day 200 kA/cm) !!

Thermonuclear scaling: derive the ratio:  $Y_{th} / E_{pinch}$  at thermonuclear pinch conditions.

- $Y_{th} = 0.5 n_i^2 \pi r_p^2 z_f \langle \sigma v \rangle \tau$

where  $\langle \sigma v \rangle$  is the thermalised fusion cross section-velocity product corresponding to the plasma temperature [149], for the lifetime of the pinch  $\tau$ .

Dividing this number by  $E_{pinch}$ :

- $Y_{th} / E_{pinch} = 0.17 \langle \sigma v \rangle / (kT) n_i \tau$  number of D-T neutrons per J of pinch energy.

# General scaling of Q as function of $n_i \tau$ at pinch temperature T

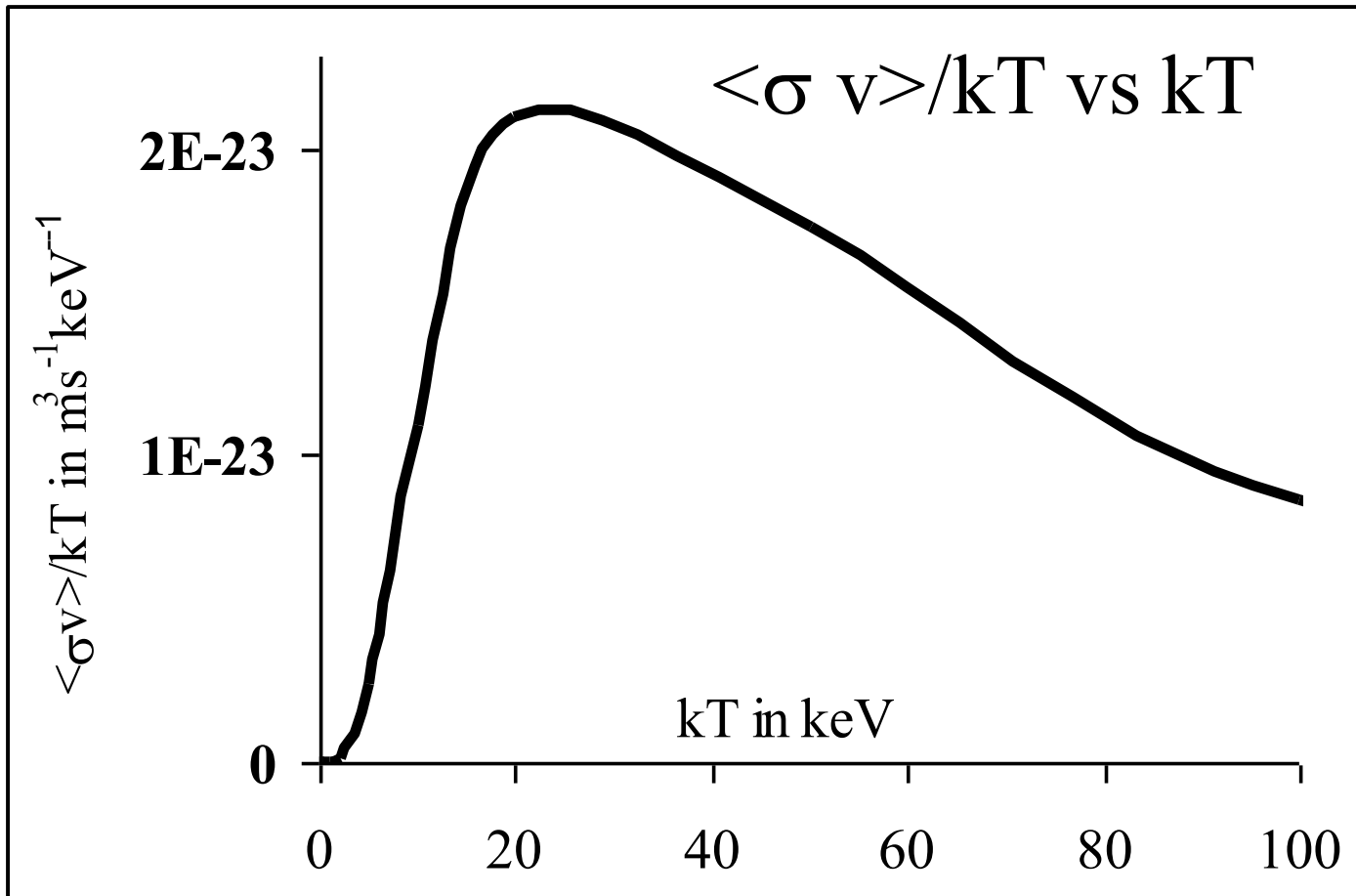
- $$\underline{E_{th}/E_0 = 240 [ \langle \sigma v \rangle / kT ] n_i \tau}$$

where  $kT$  is in keV. Here estimate  $E_{pinch} = 0.1 E_0$   
and energy of 1 D-T neutron as  $14.1 \times 10^3$  keV.

Note the Q value is a function of  $(n_i \tau)$

and the fusion x-section parameter  $[ \langle \sigma v \rangle / kT ]$

# Optimum [ $\langle\sigma v\rangle/kT$ ]



# Selecting optimum operational temperature

- From [ $\langle\sigma v\rangle/kT$ ] curve, optimum occurs at  $T = 20 \text{ keV}$  with value =  $2.1 \times 10^{-23} \text{ m}^3\text{s}^{-1}(\text{keV})^{-1}$ .
- Such a concept was expressed in seminal form by Lawson who selected 25 keV

[14. [https://en.wikipedia.org/wiki/Lawson\\_criterion](https://en.wikipedia.org/wiki/Lawson_criterion)]

[15. Lawson, J. D. (December 1955). [\*Some Criteria for a Power producing thermonuclear reactor\*](#) (PDF) (Technical report). Atomic Energy Research Establishment, Harwell, Berkshire, U. K.]

# Scaling of Q as function of $n_i \tau$ at optimum temperature of 20 keV

- $E_{th}/E_0 = 5 \times 10^{-21} n_i \tau$
- Fixing operation at this optimum temperature, the requirement for better than break-even ie  $E_{th}/E_0 > 1$  is
- $n_i \tau > 2 \times 10^{20} \text{ m}^{-3}\text{s}$  **20 keV**
- This criterion applied specifically to the DPF is comparable to the well-known Lawson's  $n_i \tau$  criterion ( $n_i \tau > 1.5 \times 10^{20}$ ). [14]

# Example

- For present-day DPF operating at  $T \sim 0.5$  keV pinch duration is  $\sim 10$ - $20$  ns per cm 'a', governed by transit time of small disturbance speed across the pinch diameter.
- At **20 keV the lifetime is around 2 ns per cm 'a'**.
- For **a = 1 m the lifetime  $\tau$  is 200 ns**.
- For  $Q > 1$ , the requirement is  **$n_i > 10^{27}$ ; close to 50 atm**
- These estimates provide guidance for numerical experiments to find an operational point for  $Q > 1$ .

# Numerical experiments found an operational thermonuclear point for $Q > 1$

$L_0$	$C_0$	$b$	$a$	$z_0$	$r_0$ m $\Omega$
18	7,000	160	155	13,500	0.1
massf	currf	massfr	currfr	Model Parameters	
0.1	0.7	0.4	0.8		
$V_0$	$P_0$	MW	A	At-1 mol-2	Operational
800,000	55,000	5	1	2	Parameters

$E_0$	RESF	$c=b/a$	$I_{peak}$	$T_p$	$v_a$	$v_s$	$v_p$	$r_{min}$	$z_{max}$	pinch $\tau$	$V_{max}$	$n_i$ pinch
kJ			kA	$10^6$ K	cm/ $\mu$ s	cm/ $\mu$ s	cm/ $\mu$ s	cm	cm	ns	kV	$10^{23}/m^3$
2.24E+12	0.062	1.032	6.2E+07	267	242	257	205	30.2	258.3	250.3	1.2E+07	6.8E+04

$Y_n$	EINP	SF	ID	$E_N$ kJ	$E_N/E_0$	$Y_{th}$	$Y_{b-t}$	$Y_n$
	%		kA/cm					
1.1E+27	45.7	1,719	4.0E+05	2.4E+12	1.09	1.1E+27	2.9E+22	1.1E+27

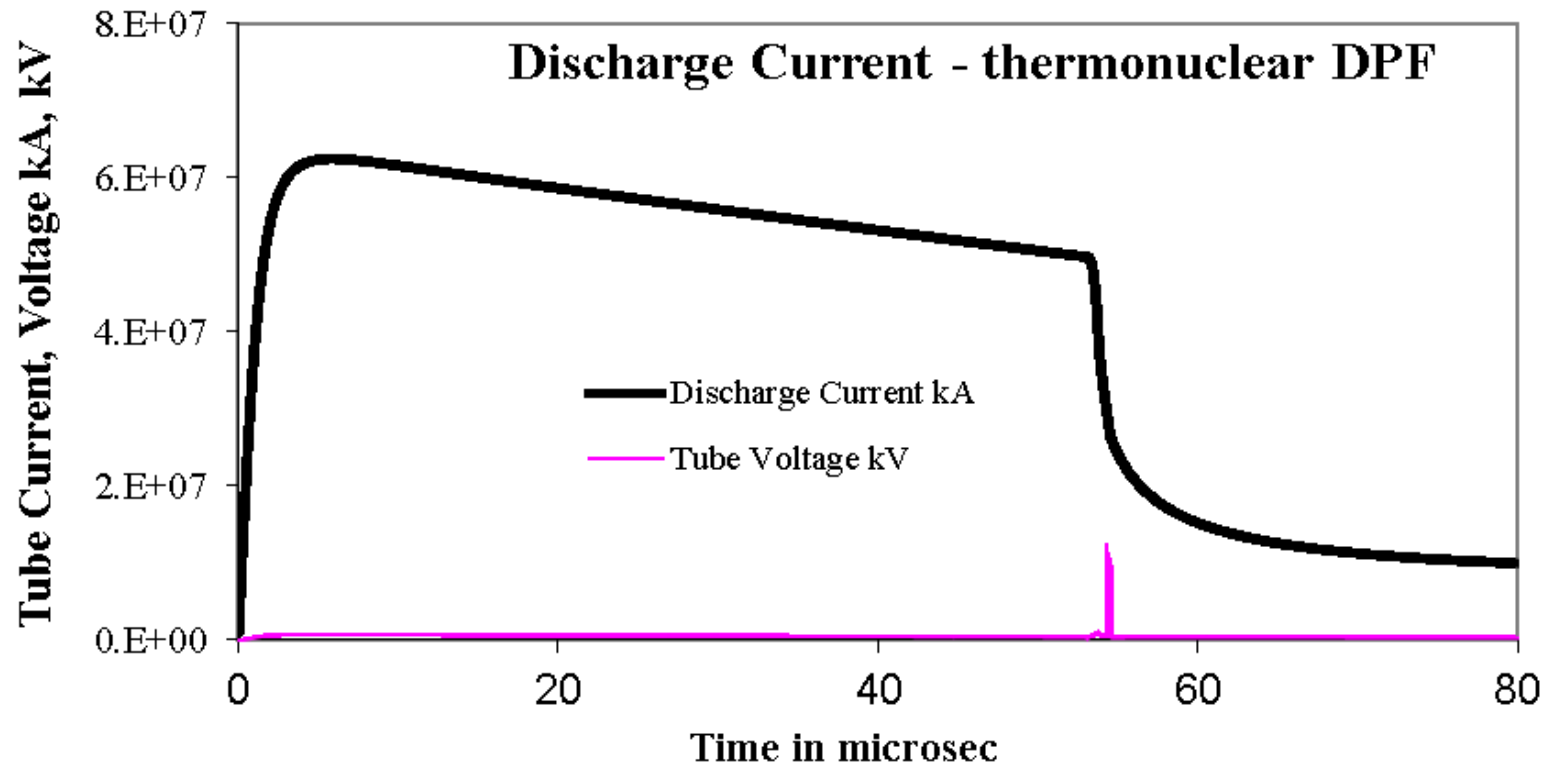
Number beam ions  $1.7 \times 10^{24}$       number of ions in radial phase original space =  $1.5 \times 10^{28}$



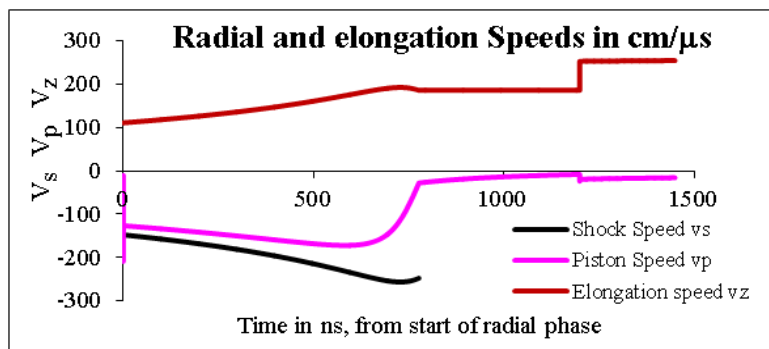
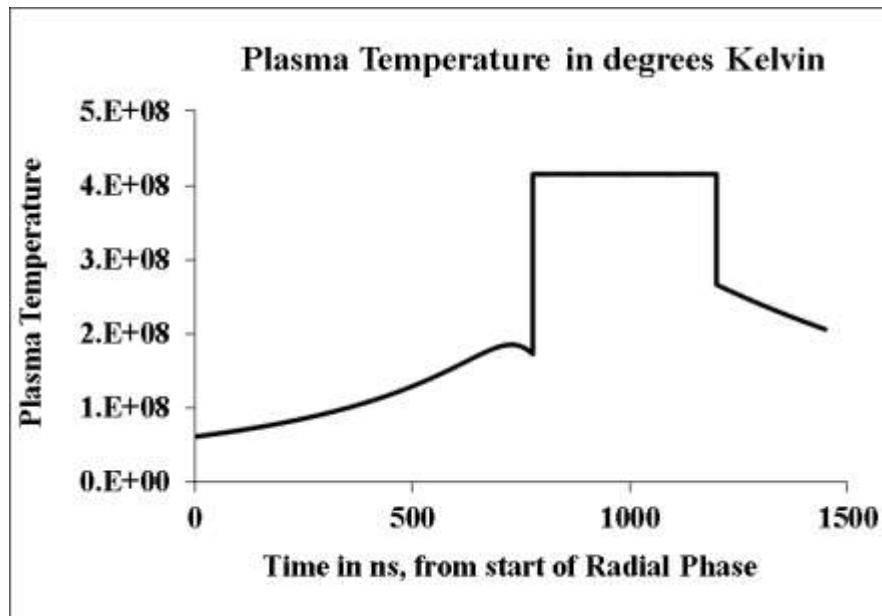
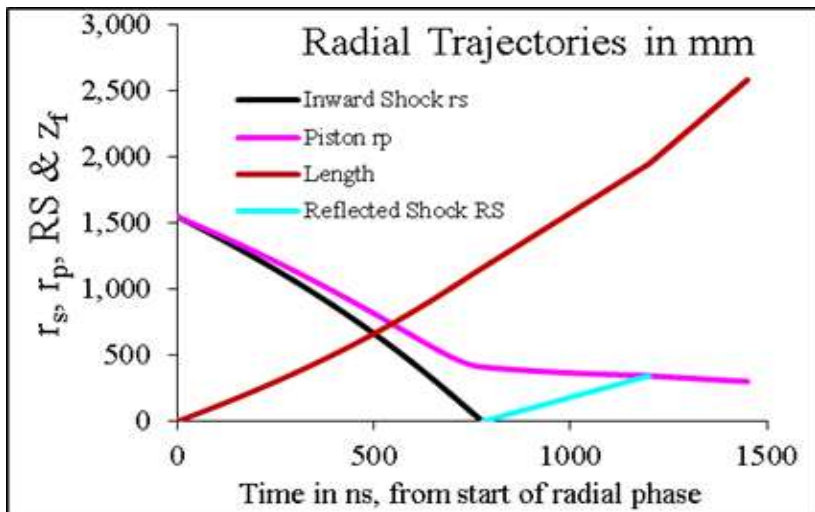
# Numerical experiment with our standard code

- The code found the following **possible point of operation**:
- Capacitor bank, **7,000  $\mu\text{F}$ , 800 MV**;
- DPF operating pressure: **70 atm**. Anode is 13.5 m long,  $a = 1.55$  m.
- The annular channel for axial phase is 5 cm with  $c = b/a = 1.03$ . Such a small channel and radius ratio is necessary to reduce the dynamic impedance of the high shock speed; otherwise the necessary high value of current will require even higher operating voltage.
- The pinch temperature is **23 keV**; pinch ion density is  $7 \times 10^{27} \text{ m}^{-3}$ , estimated duration of 250 ns.
- Thus the  **$n\tau$  is  $18 \times 10^{20}$** . The D-T neutrons are thermonuclear (99.99%)
- Yield of  $1.1 \times 10^{27}$  carrying energy of  $2.4 \times 10^{15} \text{ J}$  compared to a capacitor bank stored energy of  $2.2 \times 10^{15} \text{ J}$ ,
- Q almost 1.1.
- The current waveform and radial dynamics and temperature profiles are shown in the following figures.

# Thermonuclear DPF- current waveform

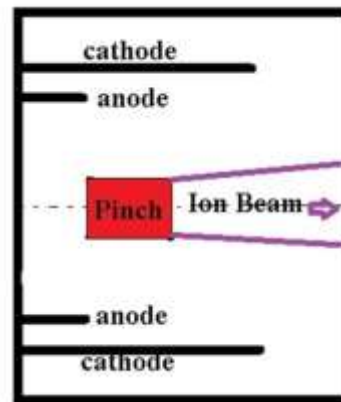


# Thermonuclear DPF - Radial Dynamics and Temperature



# IV: Summary: **B-T vs Thermonuclear**

D-T	$E_0$	$V_0$	$P_0$	a	T	U	$Y_n$	Q	Effective Q
	GJ	MV	Atm	m	keV	keV	At pinch		Ion path beyond pinch
<b>B-T (DPFQ1)</b>	<b>7.2</b>	<b>1.2</b>	<b>5.3</b>	<b>1</b>	<b>0.1</b>	<b>4.0E+03</b>	<b>6.0E+18</b>	<b>0.002</b>	<b>~1</b>
Thermo- nuclear	2.2E+06	800	72.3	1.6	23	1.2E+07	1.1E+27	1.1	



The B-T shot has  $1.3 \times 10^{22}$  ions in the beam (unreacted as computed by the code). These 5 MeV D-T ions will undergo fusion reactions within the high pressure D-T gas to raise the value of Q to greater than 1.

# Comparison: **B-T vs Thermonuclear**

- The B-T point (DPFQ1) much less extreme; though still technically far away

# DPFQ1 – technically too difficult

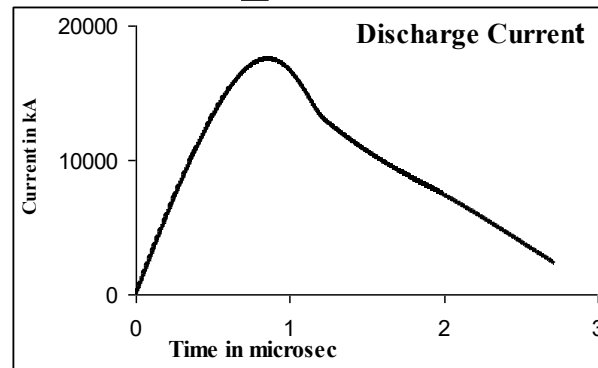
- A **technically feasible device** could be attempted first – DPF0.01

# Proposed B-t trial point – **DPF0.01**- technically feasible – to reach $Q \sim 0.01$

Lo nH	Co $\mu$ F	b cm	a cm	zo cmro	m $\Omega$
30	20	18	15	3	3
massf	currf	massfr	currfr		
0.08	0.7	0.2	0.7		
Vo kV	Po Torr	MW	A	At-1 mol-2	
<b>900</b>	<b>100</b>	5	1	2	

Note on High pressure DPF operation: ICDMP in its recent (September 2017) meeting has discussed plans to look at experimental feasibility of the concept of high pressure optimized operation of DPF

# Numerical experiments- **DPF0.01**



$E_0$	RESF	$c=b/a$	$I_{peak}$	$T_p$	$v_a$	$v_s$	$v_p$	$r_{min}$	$z_{max}$	pinch $\tau$	$V_{max}$	$n_i$ pinch
kJ			kA	$10^6$	cm/ $\mu$ s	cm/ $\mu$ s	cm/ $\mu$ s	cm	cm	ns	kV	$10^{23}/m^3$
8.10E+03	0.08	1.20	1.8E+04	2.3	11.9	27.3	18.1	4.1	23.9	282	746	31.5

$Y_n$	EINP	SF	ID	$E_N$ kJ	$E_N/E_0$	$Y_{th}$	$Y_{b-t}$	$Y_n$	$N_{ion}$ in beam
	%		kA/cm						
6.6E+13	36.1	0.5	117	0.15	2E-05	4.5E+06	6.6E+13	6.6E+13	8.4E+18



Beam ion energy is **2.5 MeV**, beyond optimum 100 keV  
 BS = reduce ion beam divergence, using pulsed magnetic field.  
**1 m long Target tube is filled with 10 atm D-T gas to interact with remnant ion beam, increase yield ~ 500x to reach Q~0.01 .**

[16.] hossein sadeghi, Reza Amrollahi et al , “High Efficiency Focus Neutron Generator”  
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# Conclusion

- Beam-target scaling at optimum beam ion energy of 100 keV suggests 1.2 MV 5 atm D-T DPF would suffice for breakeven at stored capacitor energy of 7 GJ. (**DPFQ1**)
- For comparison scaling of thermonuclear DPF is obtained, guiding numerical experiments to operational point of 800 MV 70 atm, 23 keV [achieving  $Q = 1.2$  at stored energy of 2 million GJ.]
- Comparison of the two possible operational points shows that the Beam-target  $Q \sim 1$  point is 700 times lower in operational voltage, 300,000 times lower in capacitor energy, 14 times lower in operational pressure than the thermonuclear  $Q \sim 1$  operational point.
- The Beam-target operational point (DPFQ1) is much closer to present-day DPF in every operational parameter than the thermonuclear breakeven point.
- A present-day technologically feasible point: **DPF0.01**: 900 kV (8 MJ) 100 Torr with  $Q \sim 0.01$  is proposed for initial test- (incorporates a fusion booster 10 atm D-T target tube).
- This presentation deals with **gross DPF pinch** (scalable); have not discussed structures within the pinch (such as **hot spots**) which could modify gross scaling.

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